

# Nuclear Fuel Cycle and Proliferation Monitoring

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# Outline

- Fuel cycle overview
- Used fuel reprocessing and management
- Proliferation risk of separations technology
- Examples of fuel cycle analysis
- Lattice physics modeling of fission products
- FP transport monitoring



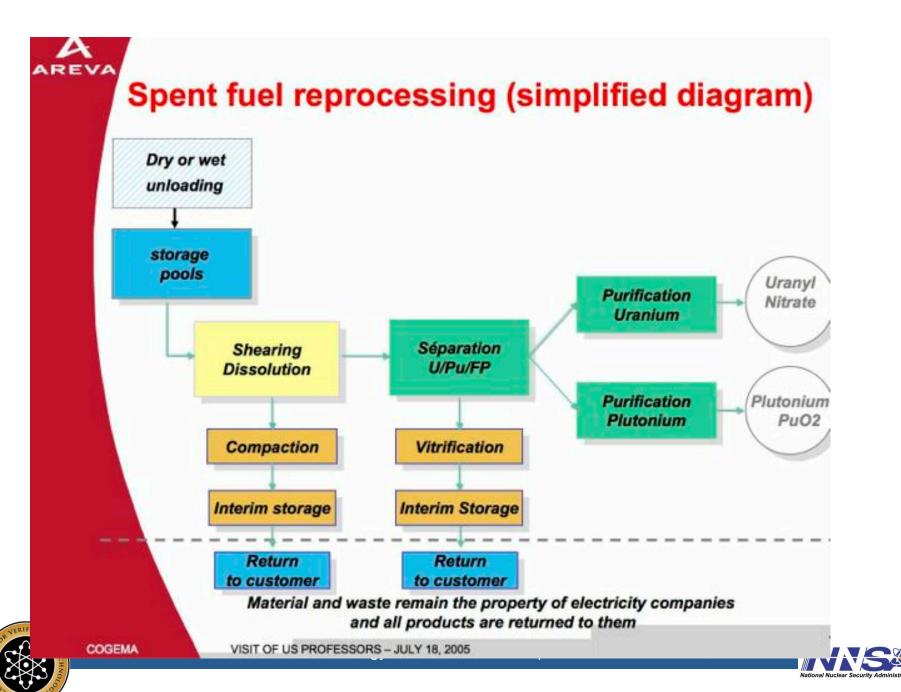


# Overview on Nuclear Fuel Cycle

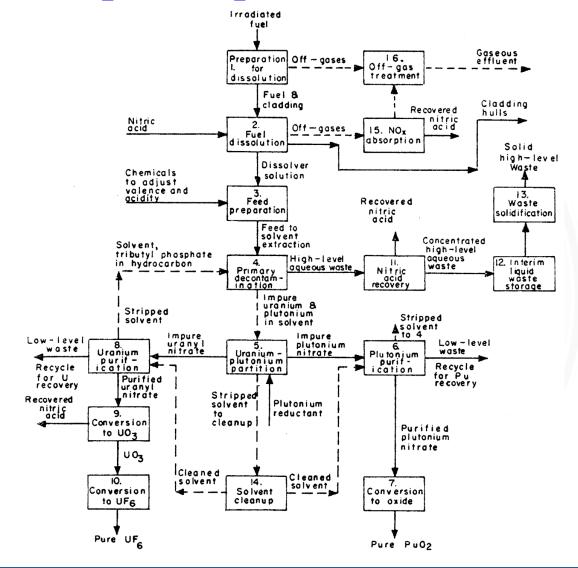
- LWRs operate with once-through closed cycles using UO<sub>2</sub> with <5.0 wt% enrichment and discharge burnup of 40~50 MWd/kgHM.
- LWR used fuel composition:
  - 3~5% fission products
  - 1% transuranics, 0.1% minor actinides [Np, Am, Cm]
  - 94~95% U.
- Current French recycling strategy:
  - Separate MA + FP + assembly metal and store vitrified logs above ground
  - Recycle (Pu+U)O<sub>2</sub> [MOX] once only in PWRs, ½ core
  - Store once-recycled MOX assemblies above ground
  - PUREX process used for reprocessing and vitrification.
- Pyroprocessing technology used for U-Pu-Zr metallic fuel from sodium-cooled fast reactor and under development for LWR fuel.







# Principal Steps of PUREX Process





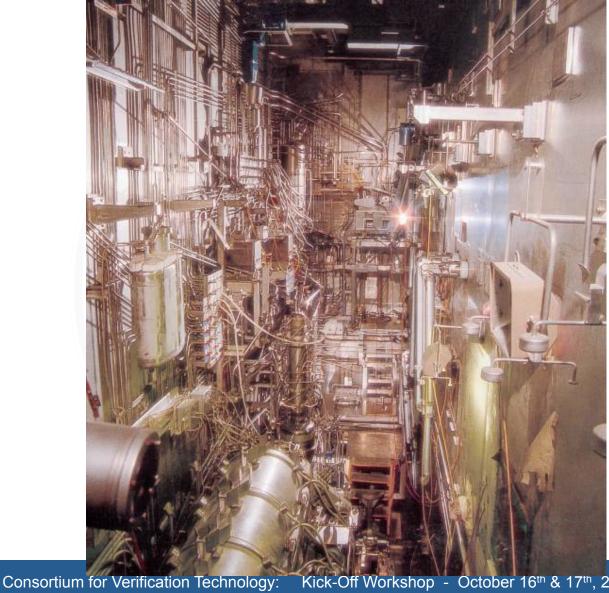


# Storage pool at La Hague





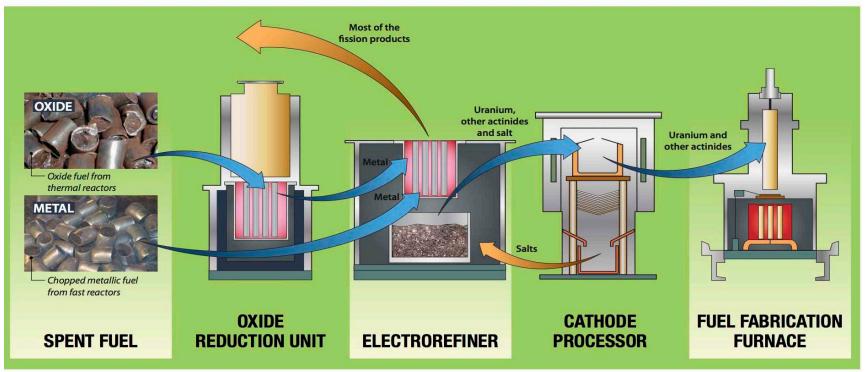
# Vitrification cell at La Hague





# Pyroprocessing Technology

- U-Pu metallic fuel is loaded at an anode in a molten salt electro-refiner.
- Electric current dissolves used fuel and plates out U-Pu on the cathode.



http://www.ne.anl.gov/About/headlines/20120723.shtml



## Proliferation Potential of Separations Technology

- Aqueous Process (PUREX)
  - Separates SNM but the material is protected via safeguards steps and physical protection in production facilities.
  - Modifying extractants and/or reductants could yield a separated stream of SNM.
- Non-aqueous Process (Pyroprocessing)
  - Batch process involving electro-refining technology in molten salt.
  - FPs (lanthanides) are lumped with SNM providing proliferation barrier.
  - Chemical composition of the salt may be adjusted to collect SNM.
  - Primarily developed for reprocessing of U-Pu-Zr metallic fuel.
- No separations technology is proliferation proof.

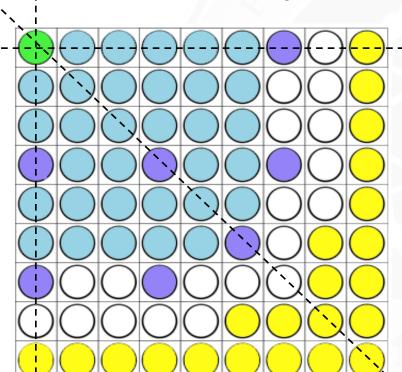




## Alternate LWR Cycle: Th-Pu MOX

#### Thorium-Based Mixed-Oxide (TMOX) Assembly

Standard 17x17 PWR assembly with 33% MOX loading

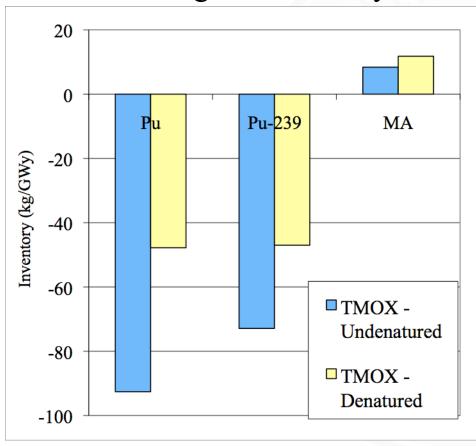


- Natural Th serves as the host for Pu in the MOX.
- TMOX not only stabilizes Pu inventory, but consumes Pu.
- Denaturing Th with <sup>238</sup>U reduces <sup>233</sup>U proliferation risk.
- $(Th,^{233}U)O_2 + Er Pin$
- (Th,Pu)O<sub>2</sub> MOX Pin
- Guide Tube
- Instrument Tube



#### TMOX Performance

#### Net Change in Inventory



- With zero <sup>239</sup>Pu production, once-through TMOX allows for a deep burn of the initial Pu loading:
  - 95% <sup>239</sup>Pu destruction
  - 70 % total Pu destruction
- Denaturing U via adding natural U deteriorates the Pu depletion capability.
- Natural U also leads to a larger MA production.

Sacrifice Pu depletion and waste reduction for proliferation resistance





# Anti-neutrino Monitoring of Reactor Fuel

• Number of anti-neutrino produced per fission depends significantly on fissionable nuclides:

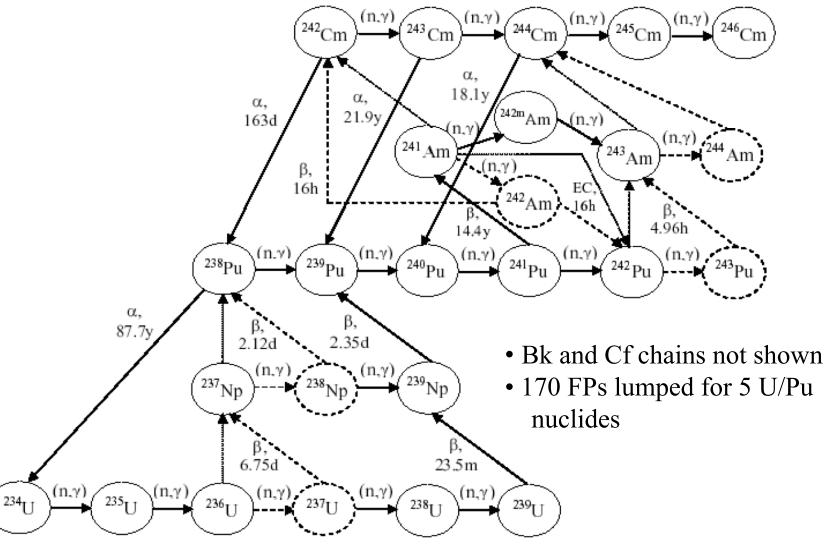
<sup>235</sup> U: 1.9 <sup>238</sup> U: 2.4 <sup>239</sup> Pu: 1.5 <sup>231</sup> Pu: 1.8

- Total anti-neutrino production rate in LWR core could be 10% different between UO<sub>2</sub> and MOX fuel.
- Total anti-neutrino production rate in SFR core could be 20% different between UO<sub>2</sub> and WG-Pu fuel.
- Need accurate data for anti-neutrino production rates for fertile Pu and higher actinides to be able to monitor fuel swaps accurately.





## Typical Decay Chain for U-Pu Fuel Cycle







## Fission Products for Radionuclide Monitoring

- NAS Report (2012) suggests increased development of radionuclide transport monitoring with the International Monitoring System.
- Lattice physics codes developed for reactor design and fuel depletion calculations focus on nuclides with large neutron absorption cross section:

<sup>135</sup> Xe: 
$$\sigma_a \simeq 2.7 \times 10^6$$
 b,  $t_{1/2} = 9.1$  h  
<sup>133</sup> Xe:  $\sigma_a \simeq 190$  b,  $t_{1/2} = 5.2$  d  
<sup>133m</sup> Xe:  $\sigma_a$  (?),  $t_{1/2} = 2.2$  d

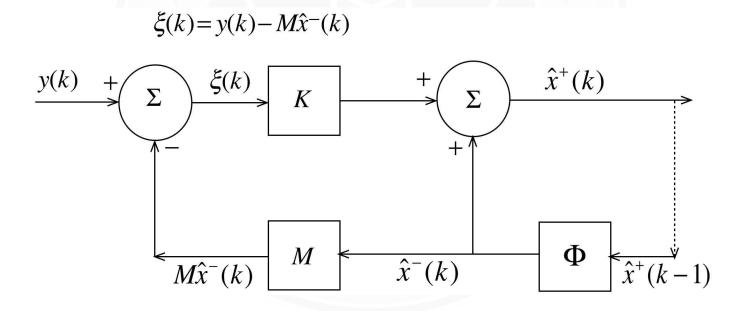
- Remaining 170 FP nuclides are lumped for 5 major U and Pu nuclides.
- ENDF-VII offers nuclear data for ~400 nuclides and TENDL-2013 includes >2000 nuclides.
- Isotopic depletion code ORIGEN represents 1119 FPs but performs one-group ENDF-VI calculations for a few pre-calculated neutron spectra.
- Need to develop accurate FP generation and transmutation modeling capability for representative fuel designs.





## Time-dependent Radionuclide Transport Modeling

- Kalman filter generates minimum-variance (optimal) estimates for timedependent system behavior combining uncertain system predictions with noisy observations.
- The filter algorithm naturally accounts for multi-modal observations.
- Recent developments of unscented Kalman filter overcomes the limitation of traditional algorithm requiring linear system representation.







## Nuclear Fuel Cycle Analysis and FP Monitoring

- Fuel cycle simulation code VISION represents fuel inventories, material flows, transmutations, economics, and other system interactions for nuclear reactor systems.
- VISION comprises several Excel I/O files built around Powersim system dynamics simulation software.
- Cyclus performs fuel cycle simulations using agent-based algorithms with agent interactions represented via dynamic resource exchange.
- Agent structure allows flexibility in implementing various fuel cycle scenarios and modes of interactions between systems.
- Proliferation markers, e.g., <sup>85</sup>Kr, <sup>129</sup>I, and <sup>3</sup>H in the off-gas stream of the PUREX process or <sup>133</sup>Xe, <sup>131</sup>I, or <sup>137</sup>Cs emission from clandestine tests could be tracked via Kalman filter and incorporated as a Cyclus agent.
- Ability to model and track FPs accurately will be useful for crosscalibration and benchmarking of IMS data.



